



# Overview of Design and R&D Activities towards a European DEMO

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on behalf of EUROfusion PPPT Department



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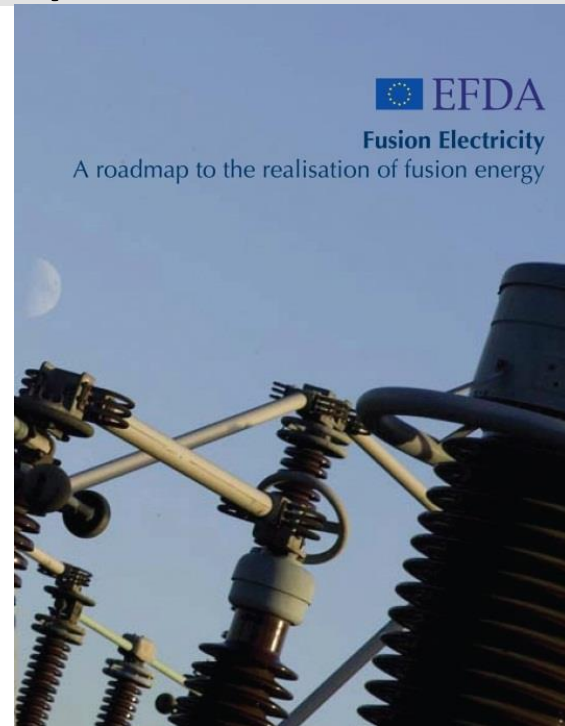


# EU Fusion Roadmap to Fusion Electricity (Update)

- An ambitious roadmap implemented by a Consortium of 29 Fusion Labs (EUROfusion)
- Distribution of resources based on priorities and on the quality of deliverables
- Support to facilities based on the joint exploitation
- Focus around [8 Programmatic Missions](#)
- Assumption in the original Roadmap:
  - ITER first plasma in early 2020's, with start of DT by 2027.

## Justification/rationale for updating DEMO part:

- ❖ Delay of ITER construction of at least 5 years : Q=10 probably achieved around mid 2030's
- ❖ General recommendation from the DEMO Stake Holders group to explore design variants longer than previously planned



### Eight Programmatic Mission

1. Plasma Operation
2. Heat Exhaust
3. Neutron resistant Materials
4. Tritium-self sufficiency
5. Safety
6. Integrated DEMO Design
7. Competitive Cost of Electricity
8. Stellarator

DEMO

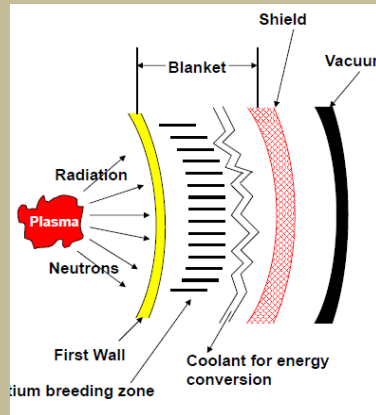


# Outstanding Technical Challenges with Gaps beyond ITER

For any further fusion step, safety, T-breeding, power exhaust, RH, component lifetime and plant availability, are important design drivers and CANNOT be compromised

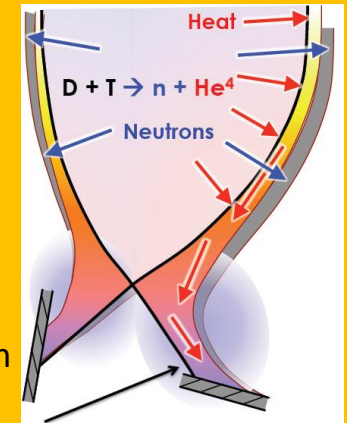
## Tritium breeding blanket

- most novel part of DEMO
- TBR >1 marginally achievable but with thin PFCs/few penetrations
- Feasibility concerns/performance uncertainties with all concepts -> R&D needed
- Selection now is premature
- ITER TBM is important



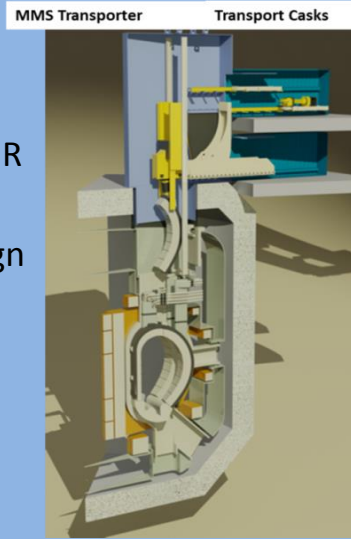
## Power Exhaust

- Peak heat fluxes near technological limits (>10 MW/m<sup>2</sup>)
- ITER solution may be marginal for DEMO
- Advanced divertor solutions may be needed but integration is very challenging
- Plans to upgrade MSTs and/or build a dedicated DTT



## Remote Maintenance

- Strong impact on IVC design
- Significant differences with ITER RM approach for blanket
- RH schemes affects plant design and layout
- Large size Hot Cell required
- Service Joining Technology R&D is urgently needed.



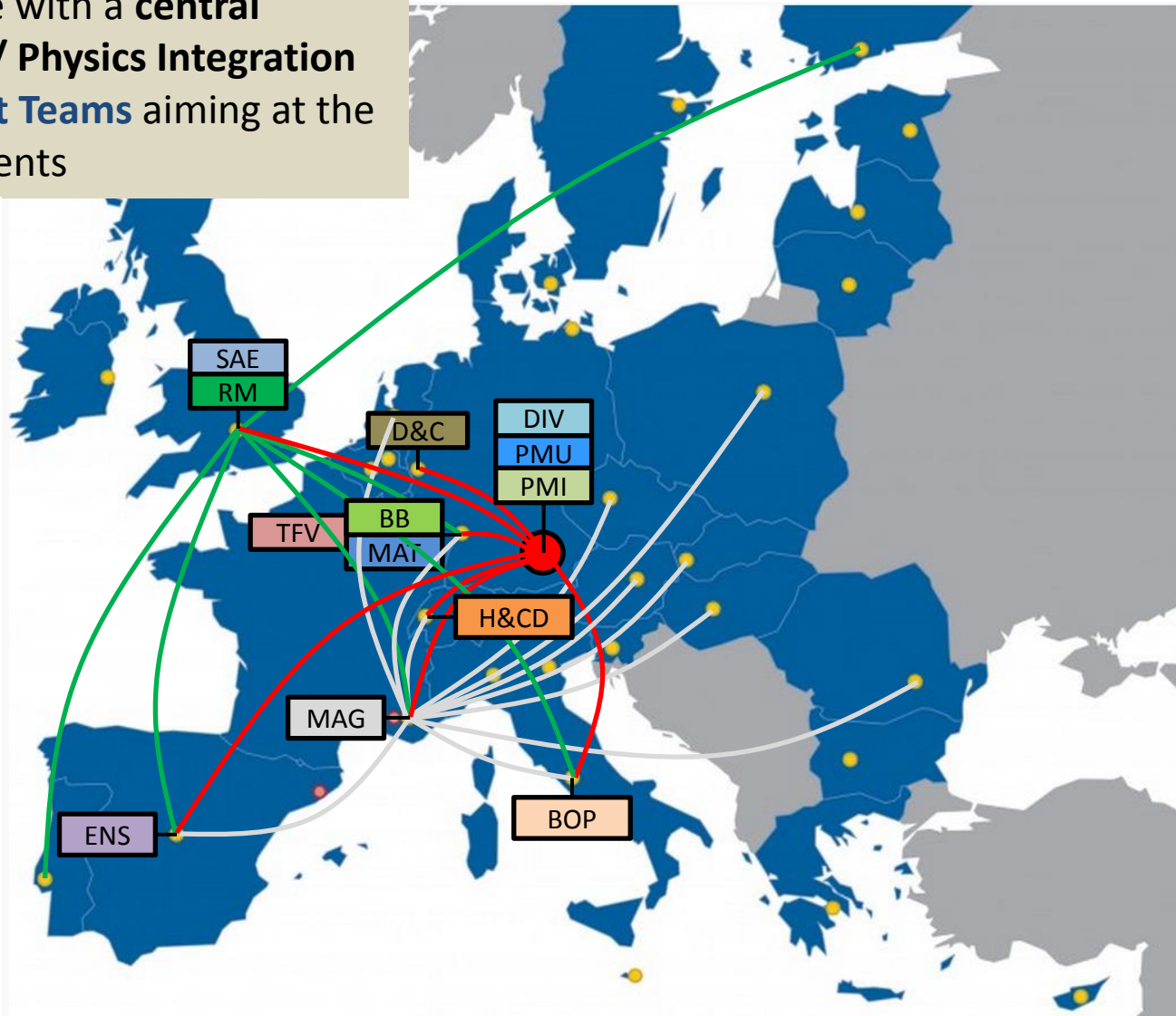
## Structural and HFF Materials

- Progressive blanket operation strategy (1<sup>st</sup> blanket 20 dpa; 2<sup>nd</sup> blanket 50 dpa)
- Embrittlement of RAFM steels and Cu-alloys at low temp. and loss of strength at ~ high temp.
- Need of structural design criteria and design codes
- N-irradiation in fission reactors selection
- Design and development of an Early Neutron Source (IFMIF-DONES)

# Organisation of Design and R&D Activities



A project-oriented structure with a **central Project Control and Design/ Physics Integration Unit** and **distributed Project Teams** aiming at the design and R&D of components





## Constraints

### ***ITER's successful operation is a prerequisite for completion of DEMO design***

- DEMO can only be built once the validity of its scenario is verified and confirmed by machine performance and operation in ITER
  - e.g. confinement, density, pedestal, self-heating for alpha-particle, divertor control, disruption control, ...
- Lesson learned from initial operation includes engineering feasibility/ component performance /infant mortality of plasma support systems (magnets, fuelling, H&CD, divertor).

### ***Availability of tritium supply***

- DEMO must breed T from day 1 and use significant amount of T (5-10 kg) for start-up.
- Current realistic forecast of civilian T supplies points to very limited quantities of T available after ITER operation.
- Operation of an intermediate device like CFETR would further stretch the problem.

### ***Political constraints***

- To justify use of public funds pressure is towards fast deployment of fusion electricity.
- Postponing the presently targeted delivery date by more than a decade bears the risk of loss of public and political interest in fusion as a solution for future energy needs.





# Revised Time Plan and Scope DEMO work

## Scope

Preparatory Phase

EFDA PPPT  
2011-2013

- Identify DEMO pre-requisites
- Identify main design and technical challenges (physics/ technology)
- Preliminary assessment technical solutions
- Prioritization of R&D to be included in the Roadmap

Pre-Conceptual Design

EUROFusion  
PPPT  
2014-2020

- Definition and analysis of initial requirements
- Preliminary design concept definition and trade-off analysis
- Identify main physics basis development needs,
- Determine critical technology development requirements (by involving more industry)
- Conduct technology and material R&D
- Concept evaluation and screening/selection of promising options

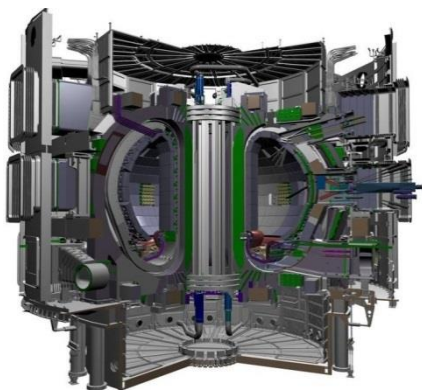
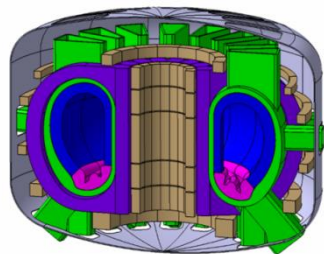
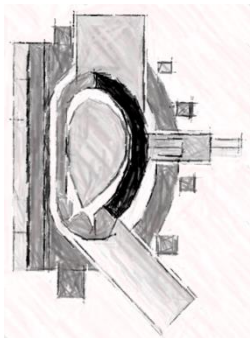
Conceptual Design

EUROFusion  
PPPT  
2021-2024

- Continue DEMO technology and material validation R&D and physics R&D
- Detailed concept definition and final trade-off analyses:
  - Divertor configuration selection and first wall protection strategy (SN/ DN)
  - Breeding blanket concept and coolant selection
  - Plasma operating scenario selection
  - H&CD mix selection

2025-2027

- Finalisation of plant concept design and reviews



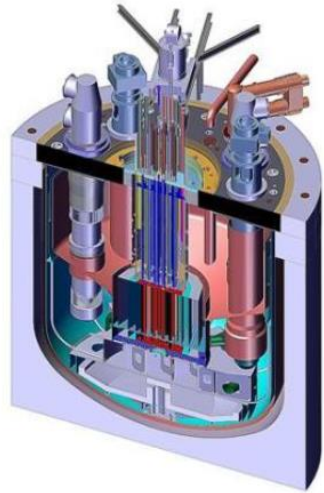
Work in Progress



# Lessons learned from Gen-IV as part of SHG Engagement

## Meetings held with GEN-IV Fission projects to gain insight into Project Execution strategies

### ASTRID :SFR Prototype GEN-IV



Integrated  
Technology  
Demonstrator  
600 MWe

F. Gauche  
(CEA)

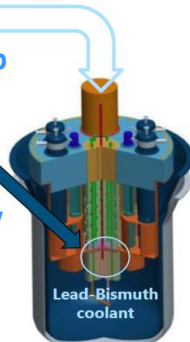
### MYRRHA: Acceleration Driven System

Reactor: Subcritical/ critical modes – 65 to 100 MW<sub>th</sub>

Accelerator: 600 MeV - 4 mA p

Spallation Source

Flexible irradiation facility



Lead-Bismuth  
coolant

- Fission projects follow pattern of evolution in each successive plant, ASTRID drawing from SuperPhenix, MYRRHA maturing from extensive test bed development.
- Design should drive R&D and not other way around.
- Fusion is a nuclear technology and as such will be assessed with full nuclear scrutiny by a regulator.
- Traceable design process with rigorous SE approach.
- Emphasis should be on maintaining proven design features (e.g., use mature technology) to minimize risks.
- Safety, reliability and maintainability should be key drivers: allow for design margins as well as redundancy within systems to ensure more fault tolerant design.
- Gen IV has leveraged impressive industry support.

1<sup>st</sup> Stake Holders Group (SHG) Meeting, 18/03/15

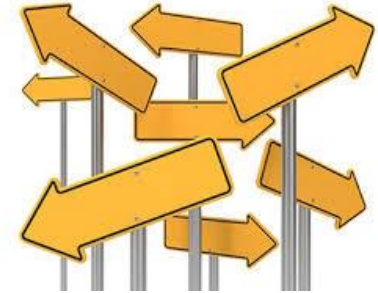
Engage experts (e.g., industry, utilities, grids, safety, licensing) to establish realistic HLRs for DEMO plant to embark on coherent conceptual design approach -> **Main outcomes: Safety, Performance and Economic viability missions.**

H. Aït Abderrahim  
(SCK-CEN)



# Design Integration / Systems Engineering Approach

- Since 2014 a traceable design process with SE approach was started to explore available DEMO design/ operation space to understand implications on technology requirements

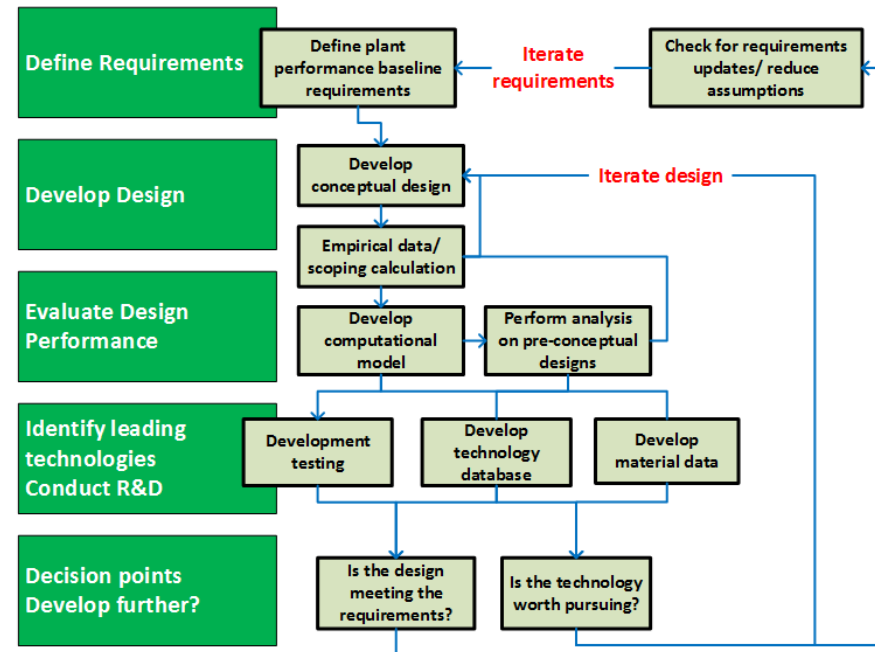


## Main Challenges

- Integration of design drivers across different projects
- Design dealing with uncertainties (physics and technology)
- High degree of system integration/ complexity/ system interdependencies
- Trade-off studies with multi-criteria optimisations, including engineering assessments.

Ensuring that R&D is focussed on resolving critical uncertainties in a timely manner and that learning from R&D is used to responsively adapt the technology strategy is crucial.

## Basic Process Flow for Conceptual Design Work



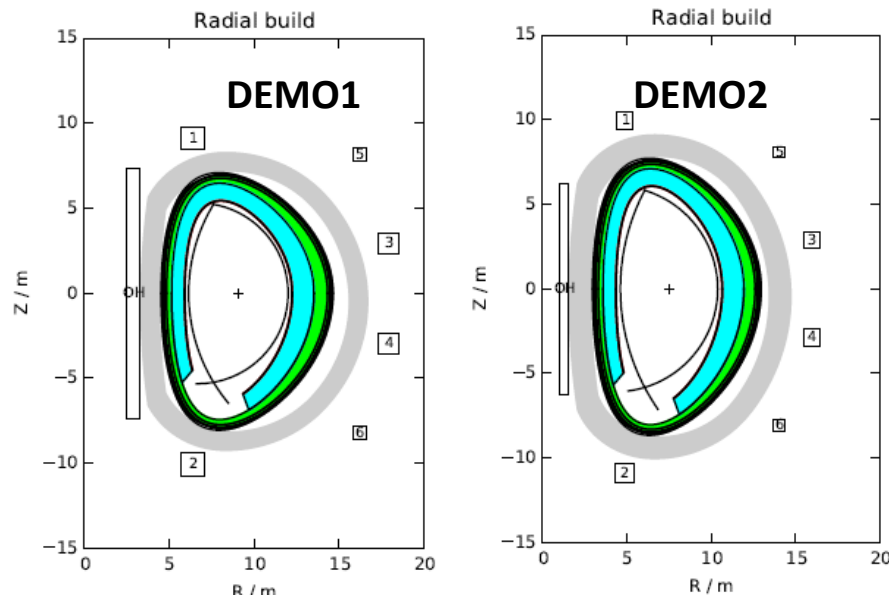




# Preliminary DEMO Design Choices under Evaluation

## Design features (near-term DEMO):

- 2000 MWth~500 Mwe
- Pulses > 2 hrs
- SN water cooled divertor
- PFC armour: W
- LTSC magnets Nb<sub>3</sub>Sn (grading)
- B<sub>max</sub> conductor ~12 T (depends on A)
- RAFM (EUROFER) as blanket structure
- VV made of AISI 316
- Blanket vertical RH / divertor cassettes
- Lifetime: starter blanket: 20 dpa (200 appm He); 2nd blanket 50 dpa; divertor: 5 dpa (Cu)



## Open Choices:

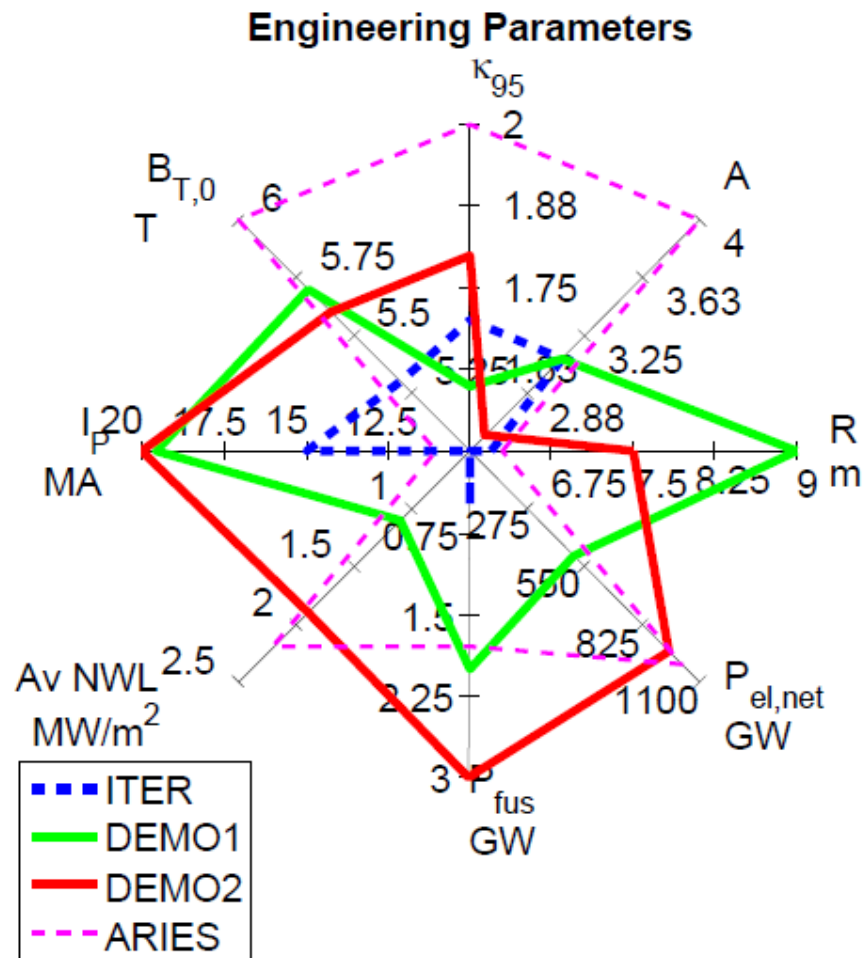
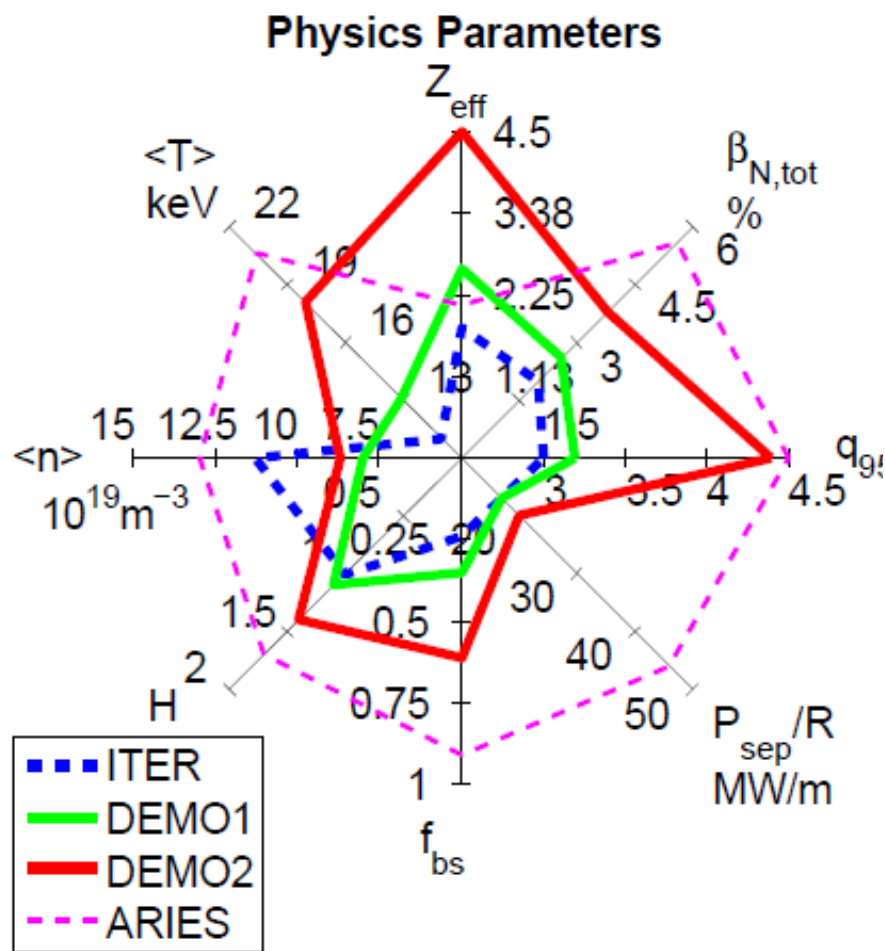
- Operating scenario
- Breeding blanket design concept selection
- Primary Blanket Coolant/ BoP
- Protection strategy first wall (e.g., limiters)
- Divertor configurations (SN, DN, advanced)
- Number of coils

	ITER	DEMO1 (2015) A=3.1	DEMO2 (2015) A=2.6
R <sub>0</sub> / a (m)	6.2 / 2.0	9.1 / 2.9	7.5 / 2.9
K <sub>95</sub> / δ <sub>95</sub>	1.7 / 0.33	1.6 / 0.33	1.8 / 0.33
A (m <sup>2</sup> ) / Vol (m <sup>3</sup> )	683 / 831	1428 / 2502	1253 / 2217
H non-rad-corr / β <sub>N</sub> (%)	1.0 / 2.0	1.0 / 2.6	1.2 / 3.8
P <sub>sep</sub> (MW)	104	154	150
P <sub>F</sub> (MW) / P <sub>NET</sub> (MW)	500 / 0	2037 / 500	3255 / 953
I <sub>p</sub> (MA) / f <sub>bs</sub>	15 / 0.24	20 / 0.35	22 / 0.61
B at R <sub>0</sub> (T)	5.3	5.7	5.6
B <sub>max</sub> conductor (T)	11.8	12.3	15.6
BB i/b / o/b (m)	0.45 / 0.45	1.1 / 2.1	1.0 / 1.9
A <sub>v</sub> NWL MW/m <sup>2</sup>	0.5	1.1	1.9



# DEMO Physics Basis / Operating Point

- Readiness of underlying physics assumptions makes the difference.
- The systems code PROCESS is being used to underpin EU DEMO design studies, and another code (SYCOMORE), is under development.



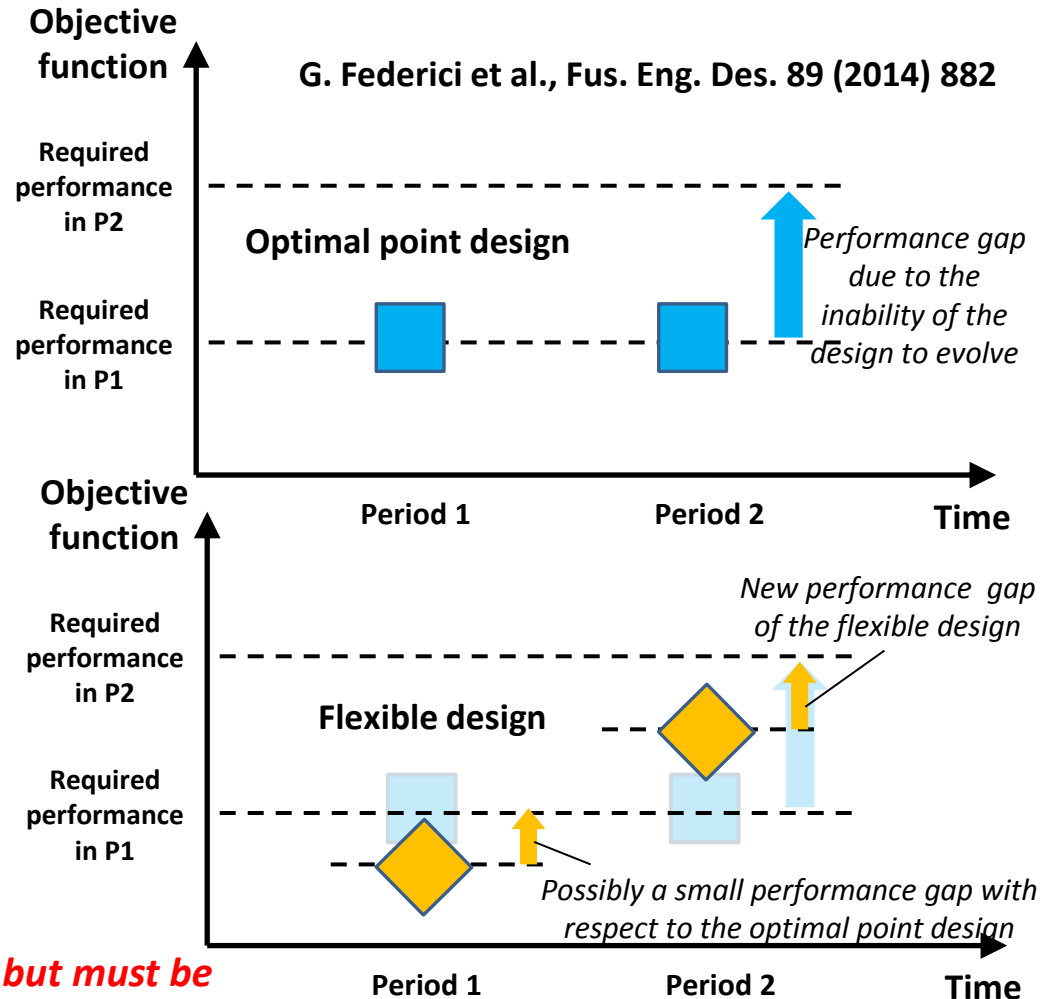


# Prospects of design staging or operation phasing

- Further develop the plasma physics, materials science, and technology while gaining experience from operating such a device and also extending its nuclear capability step by step e.g. upgrade of blanket, divertor, materials, H&CD, etc.

Traditionally, system optimisation has sought to identify an **'optimal point design'** by fixing a set of requirements and technological constraints at the start of the design => *This could result in overly constrained system unable to incorporate potential upgrades.*

However, if improvements in technology are expected over the operational lifetime of the plant, **'flexible design'** provisions should be embedded in the initial design of the system to allow the system performance to evolve with time.



**Design staging is not a one-off modification but must be carefully thought out, planned and continuously managed**

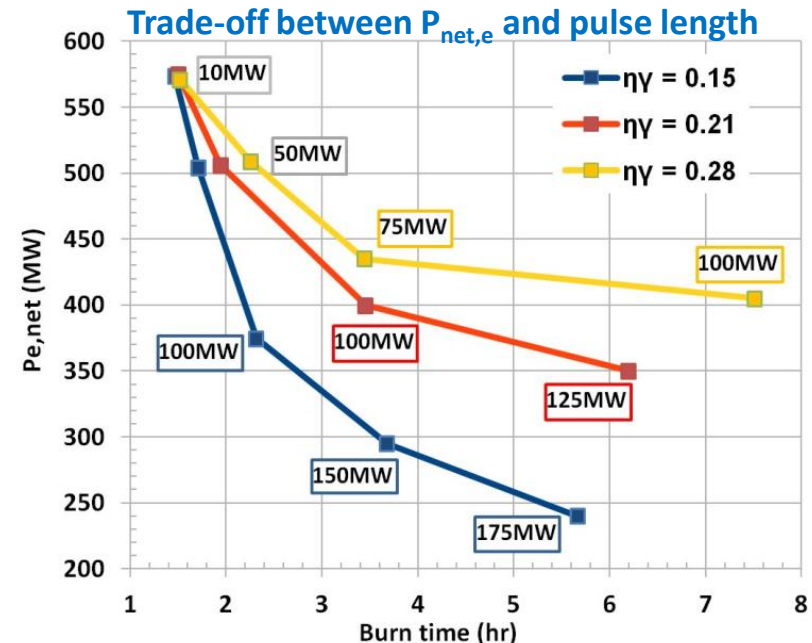
# Design flexible nuclear fusion systems is very difficult



- Staged approach and upgrades successfully followed in existing devices
- **But for a nuclear fusion reactor (like DEMO and also ITER) flexibility is much more limited**
  - ❖ A tokamak is a very complex system with multiple interfaces
  - ❖ Machine geometry will be fixed (B, I, etc.)
  - ❖ Magnetic / divertor configuration will be fixed ( $R_0$ , a, radial build, etc.)
  - ❖ Dimensional / mechanical / hydraulic Interfaces cannot be altered
  - ❖ Limited access by RH to core components through constricted ports
  - ❖ Activation of internal components / contamination
  - ❖ Changes are limited to ancillary systems e.g. fixed coolants and operating conditions

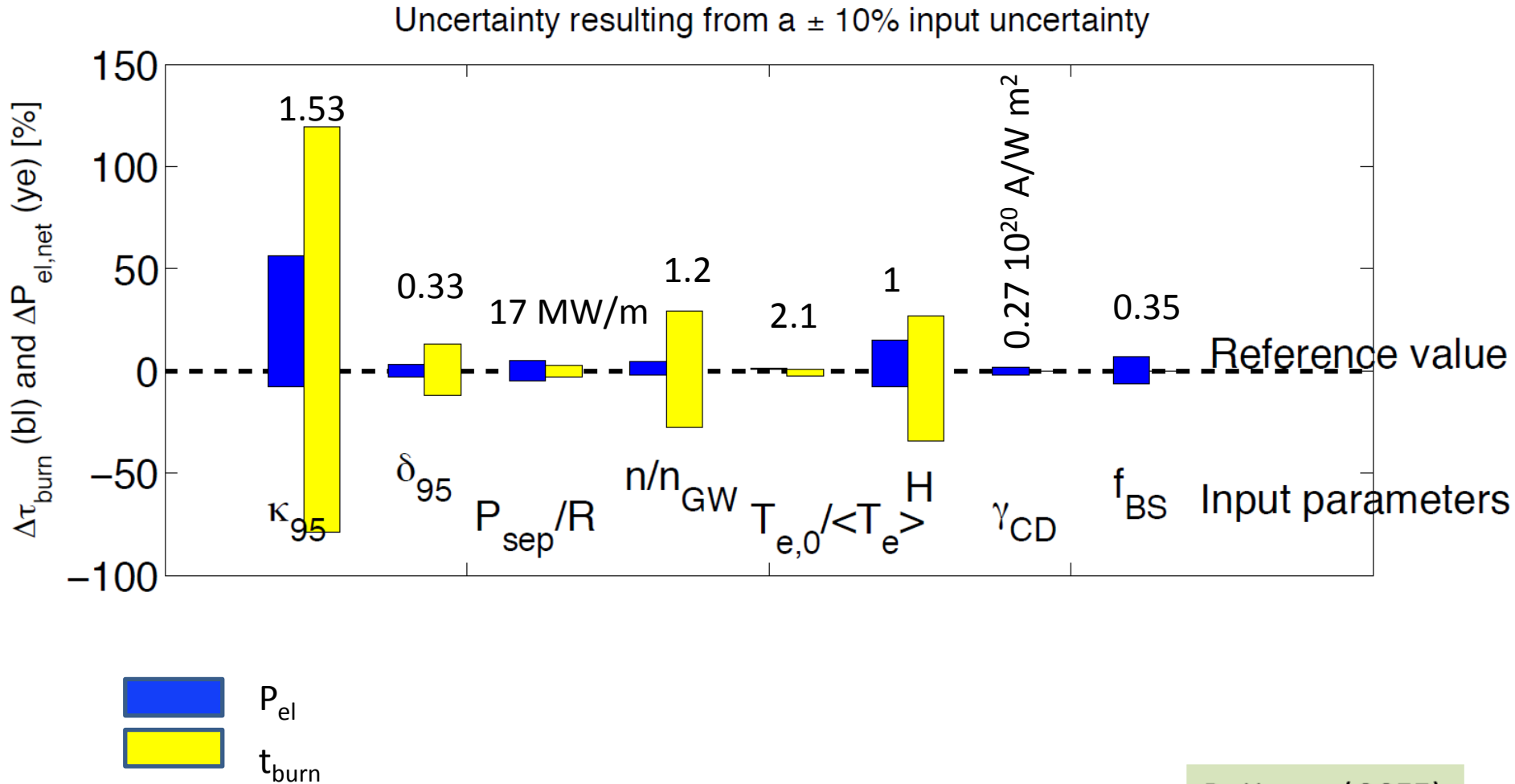
## Limited potential upgrade paths, e.g.,:

- Utilize a "starter" blanket with a higher fluence blanket upgrade from material advances
- Extension of inductive pulse by auxiliary H&CD (if  $\eta_{CD}$  can be improved, see graph)
- Improved plasma control with better diagnostics





# Point Designs "Robustness" / Uncertainties of Physics Assumptions



R. Kemp (CCFE)





# TBR Sensitivity Analysis

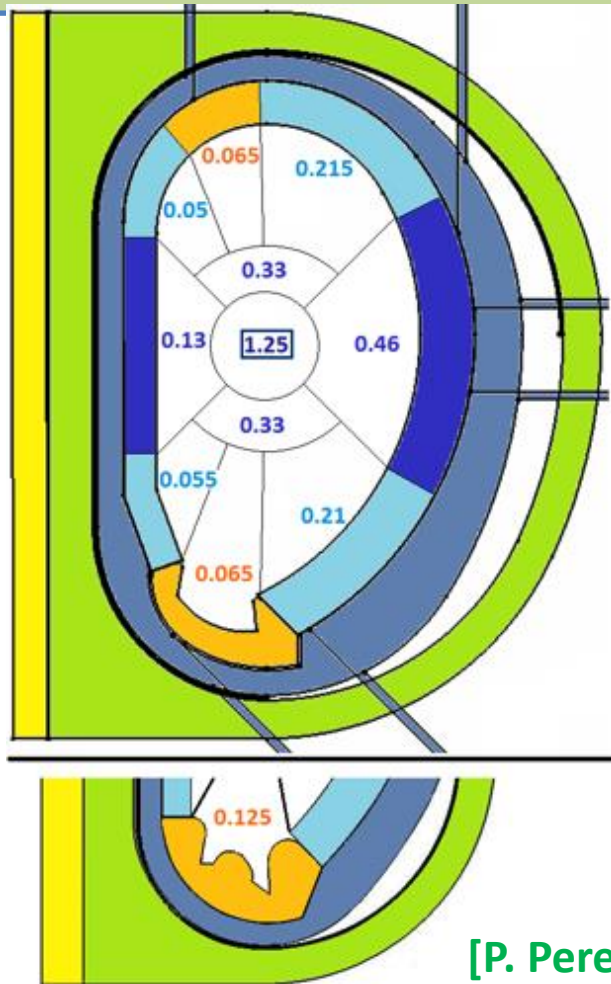
Potential Tritium breeding contributions:

Blanket design:

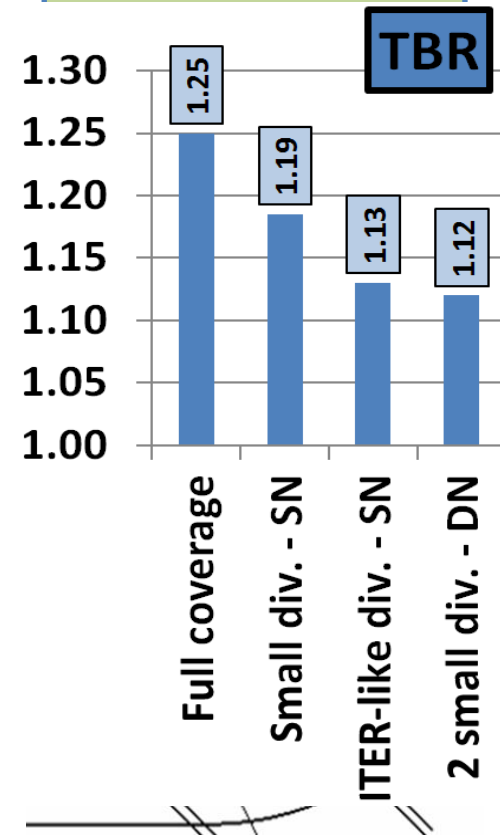
- Breeder/multiplier materials are within a box and covered by a FW.
- Box is reinforced by stiffening grids
- n-absorption by steel

Blanket size (radial thickness):

- Inb: ~80 cm / Out: ~130 cm
- Requirement: TBR ≥ 1.05
- (after integration of diagn/ H&CD)
- Configuration: About 85% of the plasma must be covered by the breeding blanket.
- Integration issue: Space for divertor, limiters, and auxiliary systems is limited.



Total TBR:



[P. Pereslavl'tsev, ISFNT12]

- Significant improvement of TBR due to reduction of divertor size.
- DN configuration with two small divertors seems possible regarding TBR.



## ❖ Japan (Broader Approach) IFERC

- joint DEMO Design Activities (DDA) to address most critical DEMO design issues investigate feasible DEMO design concepts

## ❖ China as of 2016

- **DEMO/ CFETR joint design task forces**
  - Systems codes, comparing/ benchmarking EU and CN codes
  - Divertor configuration and performance, in particular alternative divertor geometries and their potential implementation in CFETR / EU-DEMO / DTT
- **Breeding blanket research cooperation**
  - To be defined in 2016 with visit to laboratories and discussion of scope

## ❖ UCLA (DCLL)

- upgrade and use existing MaPLE facility for combined magneto-hydrodynamic (MHD) thermofluids and fluid-materials interaction experiments

## ❖ Fission Reactor Irradiation Experiment

- Collaborations to use materials test reactors outside of Europe for high fluence irradiation experiments to close gaps in the EUROFER data base

❖ **Increased involvement of industry** to ensure early attention is given to industrial feasibility, costs, nuclear safety and licensing aspects, important in design of a reactor.



- The **demonstration of electricity production ~2050** in a DEMO Fusion Power Plant is one of the priorities for the EU fusion program
- **ITER is the key facility** in this strategy and the DEMO design/R&D will benefit largely from the experience gained with ITER construction
- There are **outstanding gaps** requiring a vigorous integrated design and technology R&D (e.g., breeding blanket, divertor, Remote Handling, materials)
- Main difficulty with designing is **dealing with uncertainty**. DEMO reactor design suffers from high degree of complexity/ system Interdependencies
- Keep **reasonable flexibility** at the beginning. Trade-off studies with multi-criteria optimisations, including engineering assessments are underway and planned.
- We are developing an update of the fusion roadmap to determine possible adaptations to **minimise the impact of ITER delay** on the demonstration of fusion electricity around the middle of the century.